

Experimental Validation of Neutron Capture γ -Cascades

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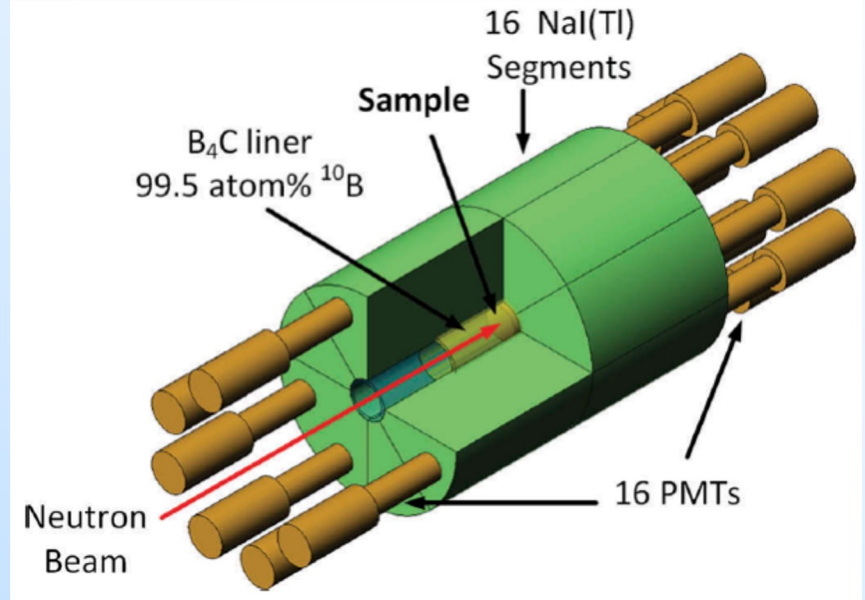
NSAC Long Range Plan Town Hall Meeting on Nuclear Structure, Reactions and Astrophysics

Nuclear Data Working Group

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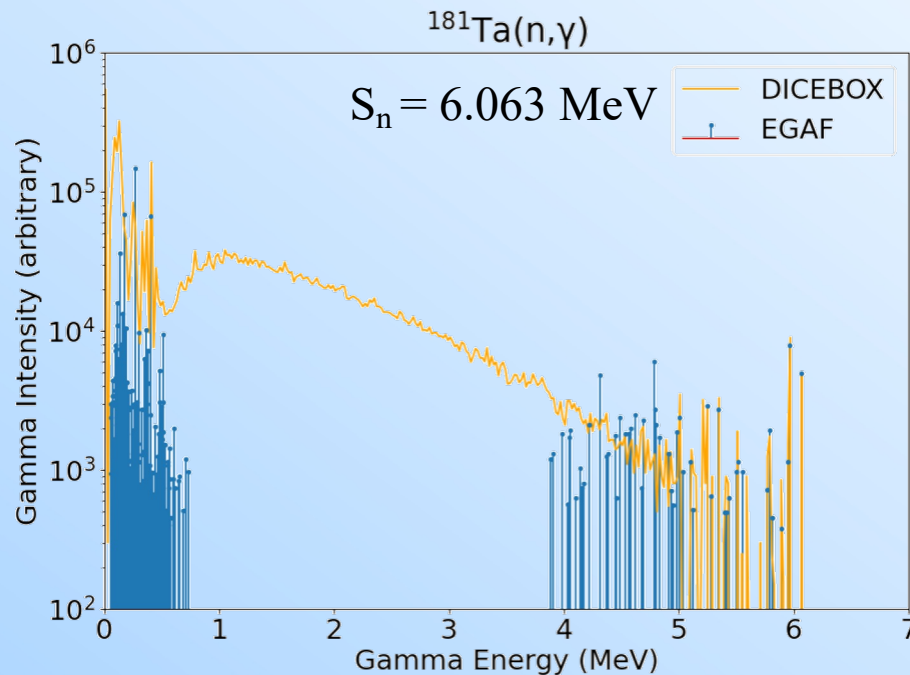
RPI γ -Multiplicity Detector

- 16 segment NaI(Tl) detector for neutron capture yield and γ -spectra measurements
 - Total volume: 20 L of NaI(Tl) surrounding the sample
 - Inside of the detector is lined (~ 1 cm) with a B_4C ceramic sleeve which is enriched 99.5 atom% in ^{10}B to absorb scattered neutrons from the sample
 - Up to 96% efficiency for detecting γ -cascades
 - **Incident neutron energies: 0.01 eV – 3 keV**



Motivation for Capture γ -spectra Measurements

- Understand γ -heating in nuclear reactors
- Characterize γ -emission spectra for **non-proliferation** applications
- Improve the current models used to **simulate γ -emission spectra**
- Increase the accuracy of nuclear data for **reactor and shielding calculations** (constrain physics models used)



DICEBOX models full γ -cascades using primary intensities from ENSDF

EGAF shows experimentally measured γ -ray lines
(does not represent the full cascade)

Neutron capture γ -cascade data is incomplete

Simulated vs. Experimental γ -spectra for $^{181}\text{Ta}(n,\gamma)$

MCNP-6.2/ENDF/B-VIII.0

Extracts γ -ray data from ENDF/B-VIII.0 files

MCNP-6.2/CGM

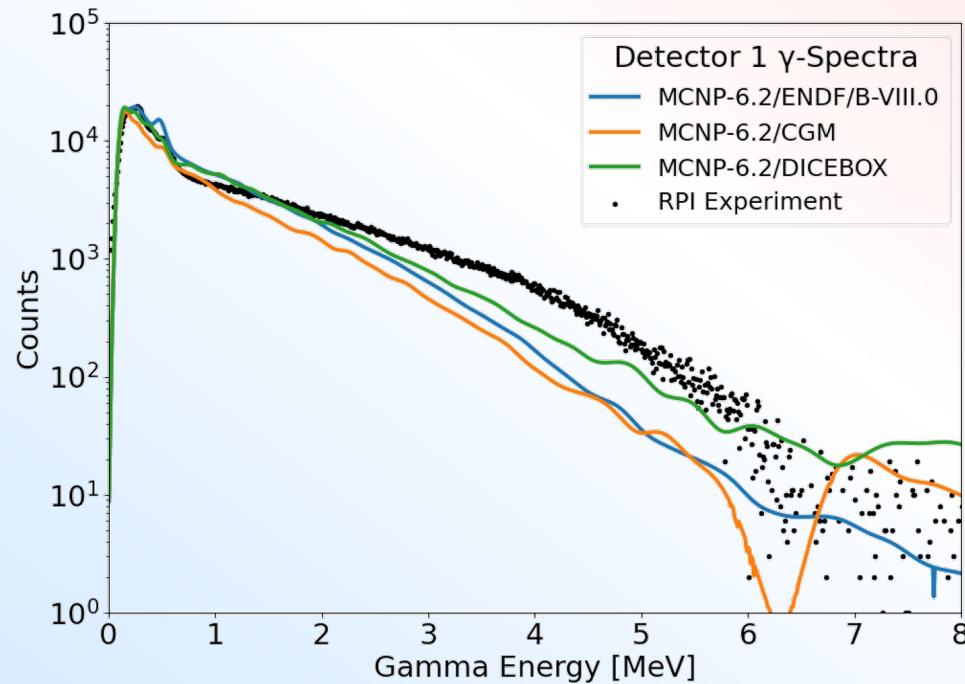
(Cascading Gamma-Ray Multiplicity)

Produces correlated secondary γ -emissions

MCNP-6.2/DICEBOX

1. γ -cascades are generated using DICEBOX
2. For each capture event, a cascade is transported through the detector geometry
3. γ -energy deposition in each of the 16 detector segments is tallied which enables event-by-event analysis including coincidence

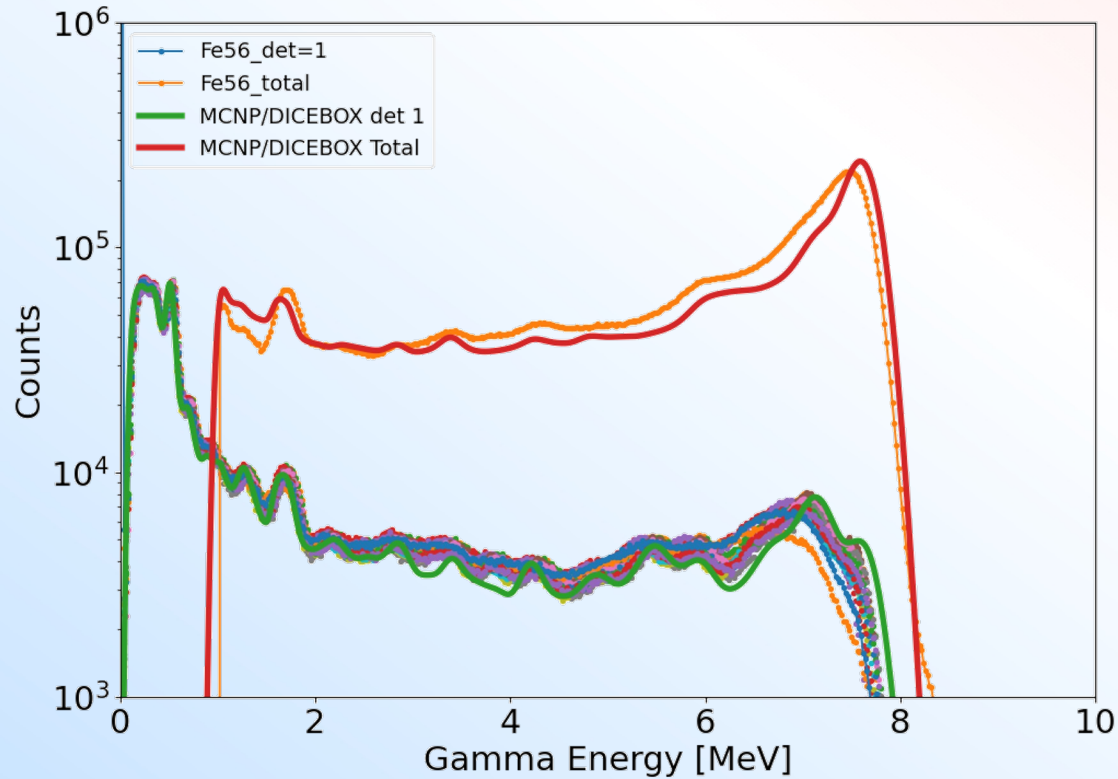
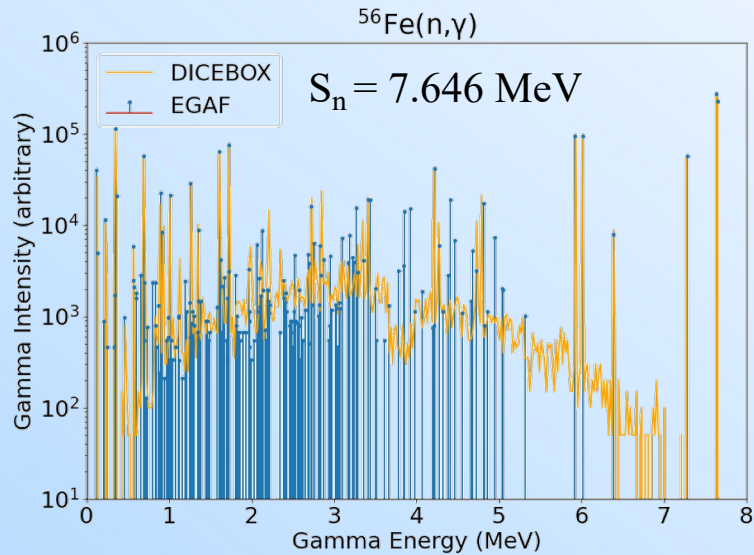
* Working with LANL to implement this capability for all MCNP users



Large discrepancies between experimental and simulated γ -spectra for $^{181}\text{Ta}(n,\gamma)$

Need a material with well-known neutron capture γ -ray data that is easy to measure

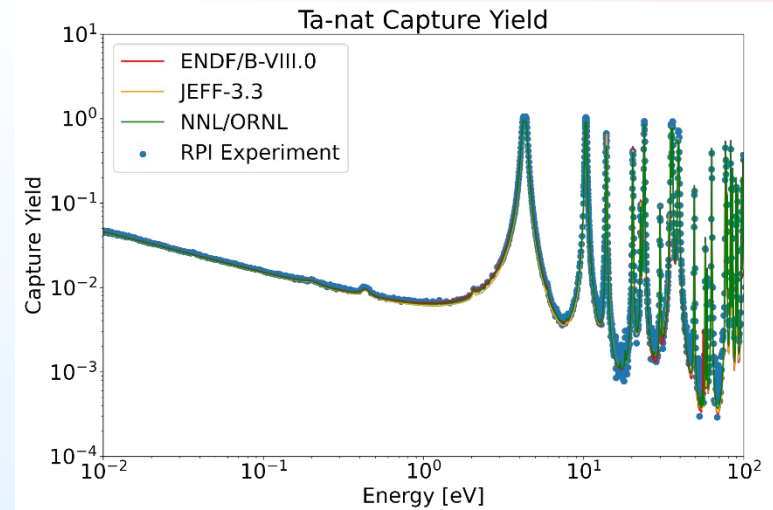
Measuring an Isotope with well-known γ -ray Data following Neutron Capture: $^{56}\text{Fe}(n,\gamma)$



Fewer discrepancies between experimental and simulated γ -spectra for $^{56}\text{Fe}(n,\gamma)$

Conclusions

- For isotopes where the neutron capture γ -cascade characteristics are well-known, the experimental system agrees with the γ -emission spectra calculated using modified simulation tools.
- RPI γ -Multiplicity Detector system can provide experimental information for isotopes with deficiencies in γ -ray data and simultaneously measure **neutron capture yield**



Future Work

- Develop a robust method for analyzing and adjusting neutron capture γ -cascade data for ^{59}Co , ^{55}Mn and other measured isotopes including ^{181}Ta , ^{238}U and ^{235}U
- Compare experimental γ -emission spectra with MCNP-6.2/DICEBOX simulations for important nuclear materials: ^{238}U and ^{235}U
 - Most interesting for reactor applications, most difficult to measure and simulate (due to the fission contribution)

